

Site (Data) : AGE

Stock of waste as at December 2004

Country: ARGENTINA

Reporting Year: 2004

Site Name: AGE

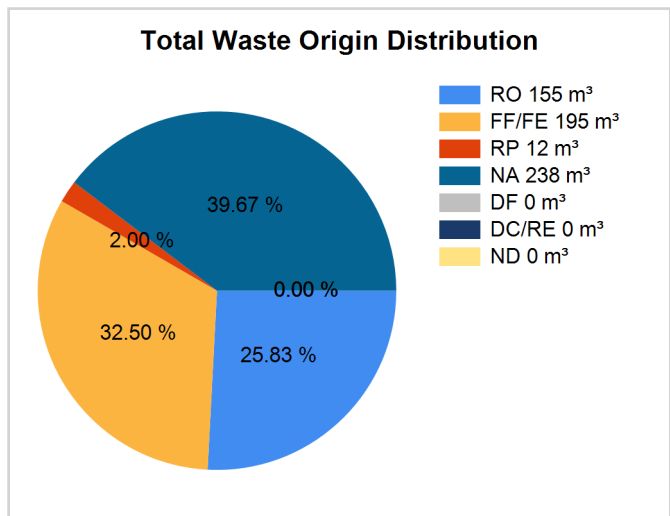
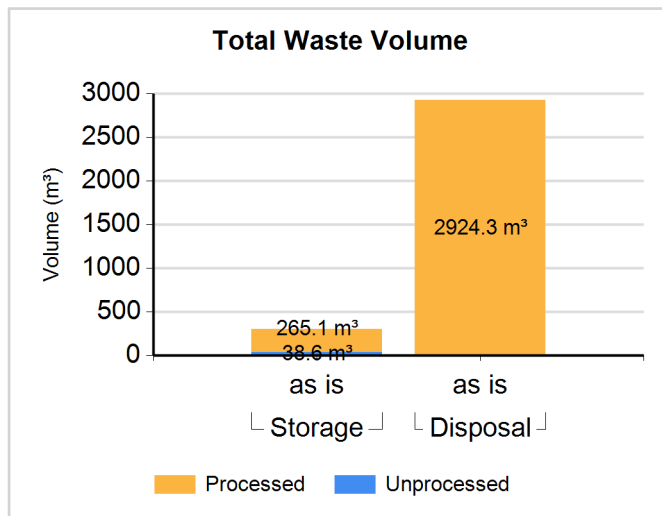
Full Name: EZEIZA WASTE MANAGEMENT AREA

Inventory Reporting Date: December 2004

Waste Matrix Used: IAEA Def.

Waste Inventory

Est=distribution is an estimate, Proc.=Is the waste processed (Yes/No)? RO=Reactor Operations, FF/FE=Fuel Fabrication/Fuel Enrichment, RP=Reprocessing, NA=Nuclear Applications,DF=Defence, DC/RE=Decommissioning/Remediation, ND=Not Determined



Note: where volume "as dispo" is provided, volume "as is" is used in the graph instead.

Waste Class: LILW-SL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-SL	Storage	N	Y	36.500	36.500	31.00	31.00	0.00	38.00	0.00	0.00	0.00
LILW-SL	Storage	Y	Y	241.500	241.500	56.00	15.00	0.00	29.00	0.00	0.00	0.00
LILW-SL	Disposal	Y	Y	2748.800	2748.800	66.00	0.00	0.00	34.00	0.00	0.00	0.00

Waste Class: LILW-LL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-LL	Storage	N	Y	2.100	2.100	0.00	50.00	0.00	50.00	0.00	0.00	0.00
LILW-LL	Storage	Y	Y	23.600	23.600	0.00	70.00	0.00	30.00	0.00	0.00	0.00
LILW-LL	Disposal	Y	Y	175.500	175.500	2.00	29.00	12.00	57.00	0.00	0.00	0.00

Site (Data) : AGE

Stock of waste as at December 2004

Country: ARGENTINA

Reporting Year: 2004

Processing - Treatment method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Compaction	N	N	Same	N
Radionuclide Separation	N	N	Same	N
Solvent Extraction	Y	N		N

Comment # 7371: Radionuclide Separation

During last year an ion exchange process was implemented in the Mo 99 production plant to separate cesium from the intermediate level waste stream. The cesium will be eluted from the column to produce cesium source for braquitherapy.

Comment # 7372: Solvent extraction

A solvent extraction process was developed to decontaminate lubricant oils used in fuel elements fabrication. This process will be implemented in rutinary operation in the near future.

Processing - Conditioning method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Cementation	Y	N		Y
Containerization	N	N	Same	N
Encapsulation	N	N	Decrease	N

Comment # 7373: Encapsulation

During 2001 to 2003 radium medical sources were encapsulated for long term storage.

Spent Sources <=30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)			c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 4GBq	Group II more than 4GBq but less than or equal 4E+4GBq	Group III more than 4E+4GBq					
	num/activity	num/activity	num/activity					
Cd-109	4			N	Y	Y	1.660E-002	2004.12
	1.660E-002							
Cf-252	1			Y	N	Y	3.520E-002	2004.12
	3.520E-002							
Co-60	43			Y	N	Y	3.820E-001	2004.12
	3.820E-001							
Co-60	128			N	Y	Y	4.940E+000	2004.12
	4.940E+000							

Site (Data) : AGE

Stock of waste as at December 2004

Country: ARGENTINA

Reporting Year: 2004

Co-60	16	4		N	Y	Y	6.220E+001	2004.12
	1.070E+001	5.150E+001						
Co-60		62		N	Y	Y	4.350E+005	2004.12
		4.350E+005						
Co-60		3	4	N	Y	Y	2.940E+005	2004.12
		1.190E+005	1.750E+005					
Cs-137		2		Y	N	Y	8.890E+002	2004.12
		8.890E+002						
Cs-137		5		N	Y	Y	8.020E+002	2004.12
		8.020E+002						
Cs-137		6		N	Y	Y	1.450E+005	2004.12
		1.450E+005						
Cs-137	40			Y	N	Y	1.250E+001	2004.12
	1.250E+001							
Cs-137	64	45		Y	N	Y	1.268E+003	2004.12
	1.280E+002	1.140E+003						
Cs-137	79			N	Y	Y	2.340E+001	2004.12
	2.340E+001							
Cs-137	84	82		N	Y	Y	2.042E+003	2004.12
	1.820E+002	1.860E+003						
Fe-55	12			N	Y	Y	6.250E+000	2004.12
	6.250E+000							
H-3	11	3		N	Y	Y	2.332E+004	2004.12
	1.764E+001	2.330E+004						
Ir-192	29			N	Y	Y	3.580E-002	2004.12
	3.580E-002							
Ir-192	1			N	Y	Y	1.320E+000	2004.12
	1.320E+000							
Kr-85	12	23		N	Y	Y	3.337E+002	2004.12
	2.670E+001	3.070E+002						

Site (Data) : AGE

Stock of waste as at December 2004

Country: ARGENTINA

Reporting Year: 2004

Pm-147	4			Y	N	Y	2.180E+000	2004.12
	2.180E+000							
Pm-147	15			N	Y	Y	5.740E+000	2004.12
	5.740E+000							
Po-210	37			N	Y	Y	2.010E-004	2004.12
	2.010E-004							
Sr-90		1		N	Y	Y	1.430E+003	2004.12
		1.430E+003						
Sr-90	72	3		N	Y	Y	6.220E+001	2004.12
	3.910E+001	2.310E+001						
Sr-90		2		N	Y	Y	2.980E+002	2004.12
		2.980E+002						

Spent Sources > 30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)		c	u	c	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 2 GBq	Group II more than 2GBq					
	num/activity	num/activity					
Am-241		3	Y	N	Y	9.380E+002	2004.12
		9.380E+002					
Am-241		7	Y	N	Y	1.300E+002	2004.12
		1.300E+002					
Am-241	22		Y	N	Y	3.890E-001	2004.12
	3.890E-001						
Am-241		2	N	Y	Y	2.461E+003	2004.12
		2.461E+003					
Am-241		16	N	Y	Y	2.510E+003	2004.12
		2.510E+003					
Am-241	33	49	N	Y	Y	6.237E+002	2004.12
	4.970E+001	5.740E+002					

Site (Data) : AGE

Stock of waste as at December 2004

Country: ARGENTINA

Reporting Year: 2004

Am-241	1471		N	Y	Y	1.480E+000	2004.12
	1.480E+000						
Ni-63	40		N	Y	Y	1.920E+001	2004.12
	1.920E+001						
Pu-238	1		N	Y	Y	8.760E-001	2004.12
	8.760E-001						
Pu-238		20	N	Y	Y	1.459E+003	2004.12
		1.459E+003					
Ra-226	731		Y	N	Y	2.330E+002	2004.12
	2.330E+002						
Ra-226	84		Y	N	Y	8.180E+000	2004.12
	8.180E+000						
Ra-226	97		N	Y	Y	5.450E+001	2004.12
	5.450E+001						
Ra-226	23		N	Y	Y	1.710E+000	2004.12
	1.710E+000						