

Site (Data) : AGE

Stock of waste as at December 2005

Country: ARGENTINA

Reporting Year: 2005

Site Name: AGE

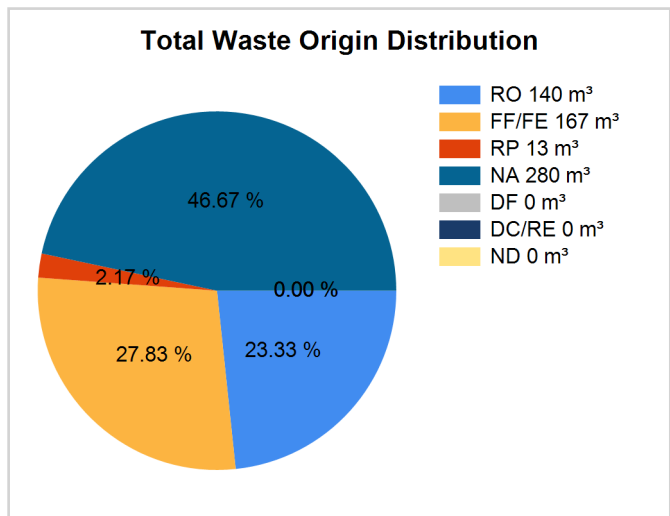
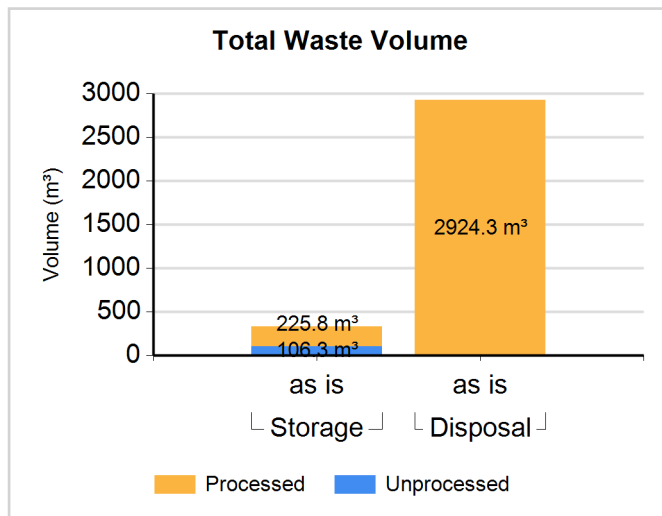
Full Name: EZEIZA WASTE MANAGEMENT AREA

Inventory Reporting Date: December 2005

Waste Matrix Used: IAEA Def.

Waste Inventory

Est=distribution is an estimate, Proc.=Is the waste processed (Yes/No)? RO=Reactor Operations, FF/FE=Fuel Fabrication/Fuel Enrichment, RP=Reprocessing, NA=Nuclear Applications,DF=Defence, DC/RE=Decommissioning/Remediation, ND=Not Determined



Note: where volume "as dispo" is provided, volume "as is" is used in the graph instead.

Waste Class: LILW-SL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-SL	Storage	N	Y	102.000	102.000	13.00	27.00	0.00	60.00	0.00	0.00	0.00
LILW-SL	Storage	Y	Y	202.800	202.800	61.00	22.00	0.00	17.00	0.00	0.00	0.00
LILW-SL	Disposal	Y	Y	2754.700	2754.700	64.00	1.00	0.00	35.00	0.00	0.00	0.00

Waste Class: LILW-LL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-LL	Storage	N	Y	4.300	4.300	0.00	28.00	0.00	72.00	0.00	0.00	0.00
LILW-LL	Storage	Y	Y	23.000	23.000	0.00	43.00	0.00	57.00	0.00	0.00	0.00
LILW-LL	Disposal	Y	Y	169.600	169.600	2.00	46.00	13.00	39.00	0.00	0.00	0.00

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Processing - Treatment method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Compaction	N	N	Same	N
Radionuclide Separation	N	N	Same	N
Solvent Extraction	Y	N		N

Comment **# 7371: Radionuclide Separation**

During last year an ion exchange process was implemented in the Mo 99 production plant to separate cesium from the intermediate level waste stream. The cesium will be eluted from the column to produce cesium source for braquitherapy.

Comment **# 7372: Solvent extraction**

A solvent extraction process was developed to decontaminate lubricant oils used in fuel elements fabrication. This process will be implemented in rutinary operation in the near future.

Processing - Conditioning method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Cementation	Y	N		Y
Containerization	N	N	Same	N
Encapsulation	N	N	Decrease	N

Comment **# 7373: Encapsulation**

During 2001 to 2003 radium medical sources were encapsulated for long term storage.

Spent Sources <=30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)			c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 4GBq	Group II more than 4GBq but less than or equal 4E+4GBq	Group III more than 4E+4GBq					
	num/activity	num/activity	num/activity					
Cd-109	4			N	Y	Y	9.500E-003	2005.12
	9.500E-003							
Cf-252	1			Y	N	Y	2.700E-002	2005.12
	2.700E-002							
Cf-252	2			N	Y	Y	7.100E-007	2005.12
	7.100E-007							
Cf-252	1			N	Y	Y	3.530E+000	2005.12
	3.530E+000							

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Co-60		4		N	Y	Y	4.150E+002	2005.12
		4.150E+002						
Co-60	43			Y	N	Y	3.330E-001	2005.12
	3.330E-001							
Co-60	166			N	Y	Y	7.130E+000	2005.12
	7.130E+000							
Co-60	17	4		N	Y	Y	8.440E+001	2005.12
	1.780E+001	6.660E+001						
Co-60		46		N	Y	Y	3.190E+005	2005.12
		3.190E+005						
Co-60		3	4	N	Y	Y	7.170E+005	2005.12
		1.060E+005	6.110E+005					
Cs-137		2		Y	N	Y	8.710E+002	2005.12
		8.710E+002						
Cs-137		7		N	Y	Y	1.250E+003	2005.12
		1.250E+003						
Cs-137		6		N	Y	Y	1.420E+005	2005.12
		1.420E+005						
Cs-137	40			Y	N	Y	1.220E+001	2005.12
	1.220E+001							
Cs-137	64	45		Y	N	Y	1.245E+003	2005.12
	1.250E+002	1.120E+003						
Cs-137	148			N	Y	Y	4.310E+001	2005.12
	4.310E+001							
Cs-137	87	121		N	Y	Y	3.702E+003	2005.12
	3.320E+002	3.370E+003						
Fe-55	14			N	Y	Y	5.060E+000	2005.12
	5.060E+000							
H-3	18	2		N	Y	Y	2.203E+004	2005.12
	3.050E+001	2.200E+004						

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Ir-192	51			N	Y	Y	6.970E-001	2005.12
	6.970E-001							
Kr-85	17	23		N	Y	Y	3.501E+002	2005.12
	3.310E+001	3.170E+002						
Pm-147	4			Y	N	Y	1.680E+000	2005.12
	1.680E+000							
Pm-147	18			N	Y	Y	6.070E+000	2005.12
	6.070E+000							
Po-210	39			N	Y	Y	3.230E-005	2005.12
	3.230E-005							
Sr-90		1		N	Y	Y	1.400E+003	2005.12
		1.400E+003						
Sr-90		3		N	Y	Y	3.040E+002	2005.12
		3.040E+002						
Sr-90	91	3		N	Y	Y	7.010E+001	2005.12
	4.750E+001	2.260E+001						

Spent Sources > 30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)		c	u	c	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 2 GBq	Group II more than 2GBq					
	num/activity	num/activity					
Am-241		3	Y	N	Y	9.380E+002	2005.12
		9.380E+002					
Am-241		7	Y	N	Y	1.300E+002	2005.12
		1.300E+002					
Am-241	22		Y	N	Y	3.890E-001	2005.12
	3.890E-001						
Am-241		2	N	Y	Y	2.461E+003	2005.12
		2.461E+003					

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Am-241		16	N	Y	Y	2.510E+003	2005.12
		2.510E+003					
Am-241	46	50	N	Y	Y	6.528E+002	2005.12
	6.820E+001	5.846E+002					
Am-241	1485		N	Y	Y	1.721E+000	2005.12
	1.721E+000						
Ni-63	44		N	Y	Y	2.068E+001	2005.12
	2.068E+001						
Pu-238	1		N	Y	Y	8.760E-001	2005.12
	8.760E-001						
Pu-238		20	N	Y	Y	1.459E+003	2005.12
		1.459E+003					
Ra-226	731		Y	N	Y	2.330E+002	2005.12
	2.330E+002						
Ra-226	84		Y	N	Y	8.180E+000	2005.12
	8.180E+000						
Ra-226	97		N	Y	Y	5.450E+001	2005.12
	5.450E+001						
Ra-226	35		N	Y	Y	2.195E+000	2005.12
	2.195E+000						