

Site (Data) : AGE

Stock of waste as at December 2006

Country: ARGENTINA

Reporting Year: 2006

Site Name: AGE

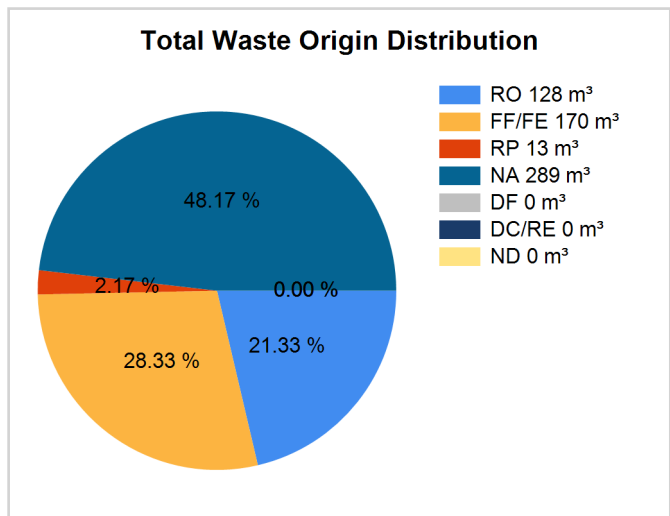
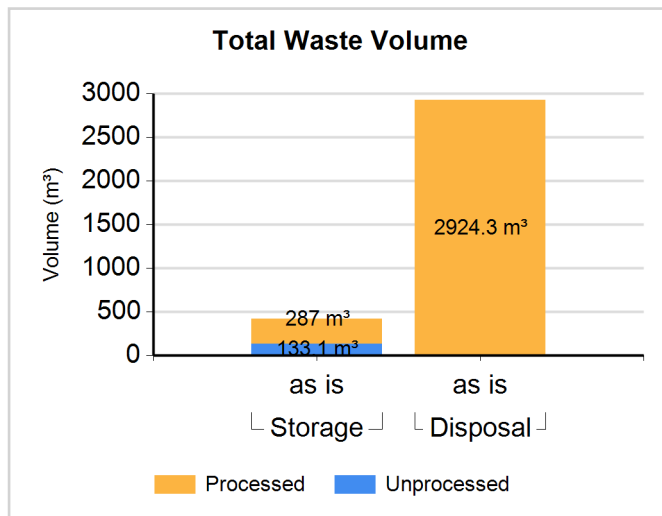
Full Name: EZEIZA WASTE MANAGEMENT AREA

Inventory Reporting Date: December 2006

Waste Matrix Used: IAEA Def.

Waste Inventory

Est=distribution is an estimate, Proc.=Is the waste processed (Yes/No)? RO=Reactor Operations, FF/FE=Fuel Fabrication/Fuel Enrichment, RP=Reprocessing, NA=Nuclear Applications,DF=Defence, DC/RE=Decommissioning/Remediation, ND=Not Determined



Note: where volume "as dispo" is provided, volume "as is" is used in the graph instead.

Waste Class: LILW-SL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-SL	Storage	N	Y	128.800	128.800	11.00	26.00	0.00	63.00	0.00	0.00	0.00
LILW-SL	Storage	Y	Y	264.000	264.000	47.00	26.00	0.00	27.00	0.00	0.00	0.00
LILW-SL	Disposal	Y	Y	2754.700	2754.700	68.00	1.00	0.00	31.00	0.00	0.00	0.00

Waste Class: LILW-LL

Waste Class Name	Location / Facility	Proc.	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-LL	Storage	N	Y	4.300	4.300	0.00	28.00	0.00	72.00	0.00	0.00	0.00
LILW-LL	Storage	Y	Y	23.000	23.000	0.00	43.00	0.00	57.00	0.00	0.00	0.00
LILW-LL	Disposal	Y	Y	169.600	169.600	2.00	46.00	13.00	39.00	0.00	0.00	0.00

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Processing - Treatment method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Compaction	N	N	Same	N
Radionuclide Separation	N	N	Same	N
Solvent Extraction	Y	N		N

Comment # 7371: Radionuclide Separation

During 2005 an ion exchange process has been implemented in the Mo 99 production plant to separate cesium from the intermediate level waste stream. The cesium will be eluted from the column to produce cesium source for brachytherapy.

Comment # 7372: Solvent extraction

A solvent extraction process was developed to decontaminate lubricant oils used in fuel elements fabrication. This process will be implemented in routine operation in the near future.

Comment # 14743: R&D: Waste Treatment

The chemical treatment of spent ion exchange resins by electromechanical methods is being studied.

Processing - Conditioning method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Cementation	Y	N		Y
Containerization	N	N	Same	N
Encapsulation	N	N	Suspended	N
Solidification	N	Y		N

Comment # 7373: Encapsulation

During 2001 to 2003 radium medical sources were encapsulated for long term storage.

Comment # 14742: Waste conditioning

The study of conditioning of LILRW in composite matrices and ceramic compounds is going to start during 2008

Spent Sources <=30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)			c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 4GBq	Group II more than 4GBq but less than or equal 4E+4GBq	Group III more than 4E+4GBq					
	num/activity	num/activity	num/activity					
Cd-109	5			N	Y	Y	9.690E-003	2006.12
	9.690E-003							
Cf-252	1			Y	N	Y	2.080E-002	2006.12
	2.080E-002							

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Cf-252	2			N	Y	Y	5.460E-007	2006.12
	5.460E-007							
Cf-252	1			N	Y	Y	2.710E+000	2006.12
	2.710E+000							
Cm-244	1			Y	N	N	3.410E-003	2006.12
	3.410E-003							
Co-60		4		N	Y	Y	3.630E+002	2006.12
		3.630E+002						
Co-60	43			Y	N	Y	2.920E-001	2006.12
	2.920E-001							
Co-60	171			N	Y	Y	7.360E+000	2006.12
	7.360E+000							
Co-60	19	4		N	Y	Y	7.650E+001	2006.12
	1.810E+001	5.840E+001						
Co-60		105		N	Y	Y	3.340E+005	2006.12
		3.340E+005						
Co-60		4	7	N	Y	Y	1.128E+006	2006.12
		1.280E+005	1.000E+006					
Cs-137		2		Y	N	Y	8.510E+002	2006.12
		8.510E+002						
Cs-137		7		N	Y	Y	1.220E+003	2006.12
		1.220E+003						
Cs-137		6		N	Y	Y	1.380E+005	2006.12
		1.380E+005						
Cs-137	40			Y	N	Y	1.190E+001	2006.12
	1.190E+001							
Cs-137	64	45		Y	N	Y	1.212E+003	2006.12
	1.220E+002	1.090E+003						
Cs-137	171			N	Y	Y	4.850E+001	2006.12
	4.850E+001							

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Cs-137	102	130		N	Y	Y	3.948E+003	2006.12
	3.580E+002	3.590E+003						
Fe-55	16			N	Y	Y	5.390E+000	2006.12
	5.390E+000							
H-3	18	2		N	Y	Y	2.083E+004	2006.12
	2.880E+001	2.080E+004						
Ir-192	51			N	Y	Y	2.300E-002	2006.12
	2.300E-002							
Ir-192	117	125		Y	N	Y	1.354E+003	2006.12
	1.140E+002	1.240E+003						
Kr-85	19	28		N	Y	Y	4.033E+002	2006.12
	3.330E+001	3.700E+002						
Pm-147	4			Y	N	Y	1.290E+000	2006.12
	1.290E+000							
Pm-147	18			N	Y	Y	4.650E+000	2006.12
	4.650E+000							
Po-210	39			N	Y	Y	5.190E-006	2006.12
	5.190E-006							
Sr-90		1		N	Y	Y	1.370E+003	2006.12
		1.370E+003						
Sr-90	95	3		N	Y	Y	6.980E+001	2006.12
	4.770E+001	2.210E+001						
Sr-90		3		N	Y	Y	2.960E+002	2006.12
		2.960E+002						

Spent Sources > 30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)		c	o	u	n	c	a	t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 2 GBq	Group II more than 2GBq									
	num/activity	num/activity									

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Am-241		3	Y	N	Y	9.380E+002	2005.12
		9.380E+002					
Am-241		7	Y	N	Y	1.300E+002	2005.12
		1.300E+002					
Am-241	22		Y	N	Y	3.890E-001	2005.12
	3.890E-001						
Am-241		2	N	Y	Y	2.461E+003	2005.12
		2.461E+003					
Am-241		17	N	Y	Y	2.610E+003	2006.12
		2.610E+003					
Am-241	58	65	N	Y	Y	1.128E+003	2005.12
	9.780E+001	1.030E+003					
Am-241	1666		N	Y	Y	2.050E+000	2006.12
	2.050E+000						
Ni-63	60	12	N	Y	Y	8.410E+001	2006.12
	2.180E+001	6.230E+001					
Pu-238	1		N	Y	Y	8.760E-001	2005.12
	8.760E-001						
Pu-238		20	N	Y	Y	1.459E+003	2005.12
		1.459E+003					
Ra-226	731		Y	N	Y	2.330E+002	2005.12
	2.330E+002						
Ra-226	84		Y	N	Y	8.180E+000	2005.12
	8.180E+000						
Ra-226	97		N	Y	Y	5.450E+001	2005.12
	5.450E+001						
Ra-226	46		N	Y	Y	2.220E+000	2006.12
	2.220E+000						