

Site (Data) : AGE

Stock of waste as at December 2008

Country: ARGENTINA

Reporting Year: 2008

Site Name: AGE

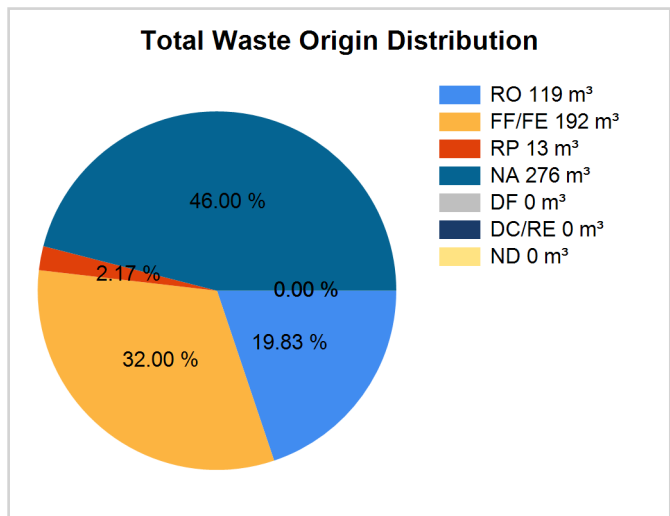
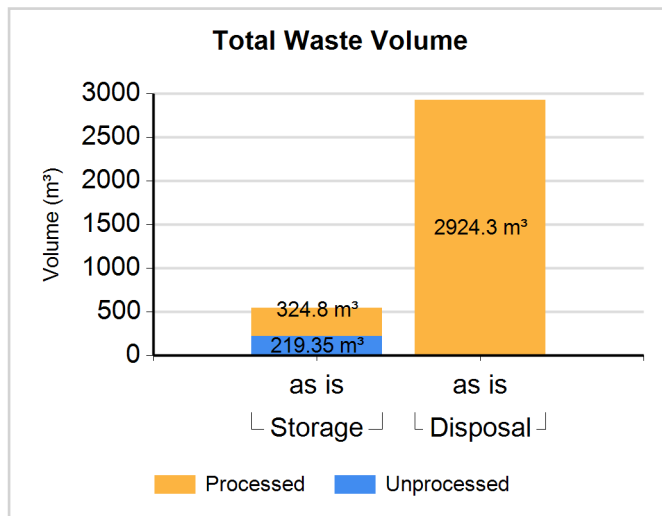
Full Name: EZEIZA WASTE MANAGEMENT AREA

Inventory Reporting Date: December 2008

Waste Matrix Used: IAEA Def.

Waste Inventory

Est=distribution is an estimate, Proc.=Is the waste processed (Yes/No)? RO=Reactor Operations, FF/FE=Fuel Fabrication/Fuel Enrichment, RP=Reprocessing, NA=Nuclear Applications,DF=Defence, DC/RE=Decommissioning/Remediation, ND=Not Determined



Note: where volume "as dispo" is provided, volume "as is" is used in the graph instead.

Waste Class: LILW-SL

Waste Class Name	Location / Facility	Proc	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-SL	Storage	N	Y	215.050	215.050	8.00	42.00	0.00	50.00	0.00	0.00	0.00
LILW-SL	Storage	Y	Y	301.800	301.800	41.00	32.00	0.00	27.00	0.00	0.00	0.00
LILW-SL	Disposal	Y	Y	2754.700	2754.700	68.00	1.00	0.00	31.00	0.00	0.00	0.00

Waste Class: LILW-LL

Waste Class Name	Location / Facility	Proc	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-LL	Storage	N	Y	4.300	4.300	0.00	28.00	0.00	72.00	0.00	0.00	0.00
LILW-LL	Storage	Y	Y	23.000	23.000	0.00	43.00	0.00	57.00	0.00	0.00	0.00
LILW-LL	Disposal	Y	Y	169.600	169.600	2.00	46.00	13.00	39.00	0.00	0.00	0.00

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Processing - Treatment method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Compaction	N	N	Same	N
Radionuclide Separation	N	N	Same	N
Solvent Extraction	Y	N		N

Comment # 7371: Radionuclide Separation

During 2005 an ion exchange process has been implemented in the Mo 99 production plant to separate cesium from the intermediate level waste stream.

Comment # 14743: R&D: Waste Treatment

The chemical treatment of spent ion exchange resins by electromical methods is being studied

Processing - Conditioning method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Cementation	Y	N		Y
Containerization	N	N	Same	N
Encapsulation	N	N	Suspended	N
Solidification	N	Y		N

Comment # 7373: Encapsulation

During 2001 to 2003 radium medical sources were encapsulated for long term storage.

Comment # 14742: Waste conditioning

The study of conditioning of LILRW in composite matrices and ceramic compounds has started.

Spent Sources <=30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)			c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 4GBq	Group II more than 4GBq but less than or equal 4E+4GBq	Group III more than 4E+4GBq					
	num/activity	num/activity	num/activity					
Cd-109	7			N	Y	Y	3.300E-003	2008.12
	3.300E-003							
Cf-252	1			Y	N	Y	1.200E-002	2008.12
	1.200E-002							
Cf-252	9			N	Y	Y	1.300E-001	2008.12
	1.300E-001							

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Cf-252	1			N	Y	Y	1.600E+000	2008.12
	1.600E+000							
Cm-244	1			Y	N	N	3.200E-003	2008.12
	3.200E-003							
Co-60	43			Y	N	Y	2.200E-001	2008.12
	2.200E-001							
Co-60	181	25		N	Y	Y	2.300E+005	2008.12
	6.400E+000	2.300E+005						
Co-60	20	4		N	Y	Y	6.000E+001	2008.12
	1.500E+001	4.500E+001						
Co-60		113		N	Y	Y	5.300E+005	2008.12
		5.300E+005						
Co-60		6	9	N	Y	Y	1.040E+006	2008.12
		1.700E+005	8.700E+005					
Co-60		6		N	Y	Y	3.400E+002	2008.12
		3.400E+002						
Cs-137		7		N	Y	Y	1.200E+003	2008.12
		1.200E+003						
Cs-137		6		N	Y	Y	1.300E+005	2008.12
		1.300E+005						
Cs-137		2		Y	N	Y	8.100E+000	2008.12
		8.100E+000						
Cs-137	40			Y	N	Y	1.100E+001	2008.12
	1.100E+001							
Cs-137	64	45		Y	N	Y	1.220E+003	2008.12
	1.200E+002	1.100E+003						
Cs-137	275			N	Y	Y	6.100E+001	2008.12
	6.100E+001							
Cs-137	117	152		N	Y	Y	4.070E+003	2008.12
	3.700E+002	3.700E+003						

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Cs-137			1	N	Y	Y	1.800E+005	2008.12
			1.800E+005					
Fe-55	20			N	Y	Y	5.700E+000	2008.12
	5.700E+000							
H-3	18	2		N	Y	Y	1.903E+004	2008.12
	2.600E+001	1.900E+004						
Ir-192	288			N	Y	Y	3.700E-002	2008.12
	3.700E-002							
Ir-192	242			Y	N	Y	1.400E+000	2008.12
	1.400E+000							
Kr-85	22	31		N	Y	Y	9.437E+003	2008.12
	3.700E+001	9.400E+003						
Pm-147	4			Y	N	Y	7.700E-001	2008.12
	7.700E-001							
Pm-147	18			N	Y	Y	2.800E+000	2008.12
	2.800E+000							
Po-210	39			N	Y	Y	2.200E-008	2008.12
	2.200E-008							
Sr-90		1		N	Y	Y	1.300E+003	2008.02
		1.300E+003						
Sr-90	112	3		N	Y	Y	2.780E+001	2008.12
	5.800E+000	2.200E+001						
Sr-90		3		N	Y	Y	2.800E+002	2008.12
		2.800E+002						

Spent Sources > 30 years in Storage

Nuclide	Number of Sources/Total Activity of Sources (GBq)		c	o	n	d	c	a	t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 2 GBq	Group II more than 2GBq									
	num/activity	num/activity									

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Am-241		3	Y	N	Y	9.400E+002	2008.12
		9.400E+002					
Am-241		7	Y	N	Y	1.300E+002	2008.12
		1.300E+002					
Am-241	22		Y	N	Y	3.900E-001	2008.12
	3.900E-001						
Am-241		2	N	Y	Y	2.500E+003	2008.12
		2.500E+003					
Am-241		17	N	Y	Y	2.600E+003	2008.12
		2.600E+003					
Am-241	96	67	N	Y	Y	1.290E+003	2008.12
	1.900E+002	1.100E+003					
Am-241	3243		N	Y	Y	6.400E+003	2008.12
	6.400E+003						
Am-241		12	N	Y	Y	1.300E+002	2008.12
		1.300E+002					
Am-241		1	N	Y	Y	1.100E+002	2008.12
		1.100E+002					
Ni-63	66	12	N	Y	Y	9.000E+001	2008.12
	2.800E+001	6.200E+001					
Pu-238	1		N	Y	Y	8.700E-001	2008.12
	8.700E-001						
Pu-238		20	N	Y	Y	1.500E+003	2008.02
		1.500E+003					
Ra-226	731		Y	N	Y	2.300E+002	2008.02
	2.300E+002						
Ra-226	84		Y	N	Y	8.200E+000	2008.12
	8.200E+000						
Ra-226	97		N	Y	Y	5.500E+001	2008.12
	5.500E+001						

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Ra-226	81		N	Y	Y	2.500E+000	2008.12
	2.500E+000						