

Site (Data) : Püspökszil

Stock of waste as at December 2005

Country: HUNGARY

Reporting Year: 2005

Site Name: Püspökszil

Full Name: Püspökszilágy LILW Repository

Inventory Reporting Date: December 2005

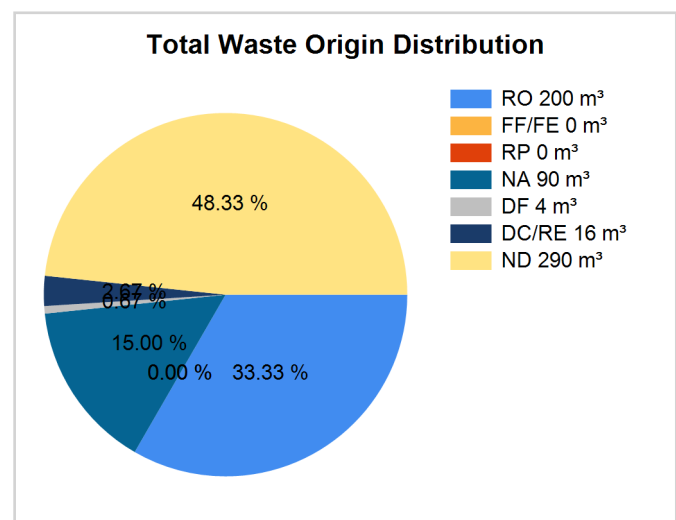
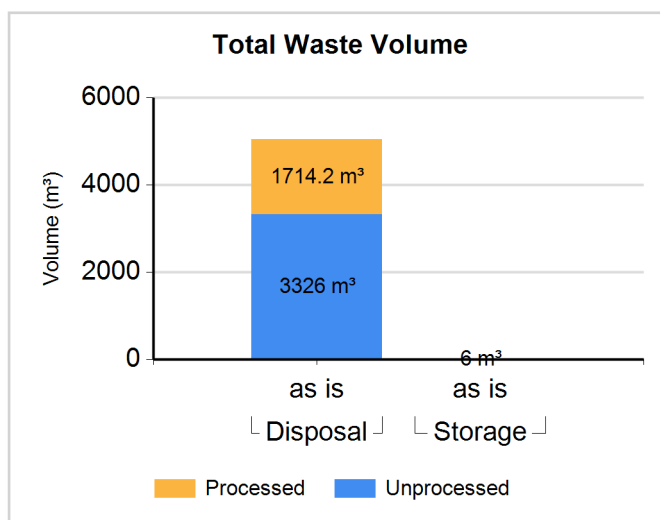
Waste Matrix Used: IAEA Def.

Comment # 339: Percentage of Capacity Used

The percentage of disposal facility capacity used takes in to consideration the volume of waste plus losses due to voids, buffer and backfill materials

Waste Inventory

Est=distribution is an estimate, Proc.=Is the waste processed (Yes/No)? RO=Reactor Operations, FF/FE=Fuel Fabrication/Fuel Enrichment, RP=Reprocessing, NA=Nuclear Applications,DF=Defence, DC/RE=Decommissioning/Remediation, ND=Not Determined



Note: where volume "as dispo" is provided, volume "as is" is used in the graph instead.

Waste Class: LILW-SL

Waste Class Name	Location / Facility	Proc	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-SL	Disposal	N	N	1330.000	1330.000	20.00	0.00	0.00	20.00	2.00	8.00	50.00
LILW-SL	Disposal	Y	N	685.000	685.000	80.00	0.00	0.00	0.00	0.00	0.00	20.00

Waste Class: LILW-LL

Waste Class Name	Location / Facility	Proc	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
LILW-LL	Storage	N	N	6.000	6.000	0.00	0.00	0.00	50.00	0.00	0.00	50.00
LILW-LL	Disposal	N	N	1996.000	1996.000	20.00	0.00	0.00	20.00	2.00	8.00	50.00
LILW-LL	Disposal	Y	N	1029.000	1029.000	80.00	0.00	0.00	0.00	0.00	0.00	20.00

Waste Class: HLW

Waste Class Name	Location / Facility	Proc	Est.	Volume "as is" (m³)	Volume "as dispo" (m³)	RO %	FF/FE %	RP %	NA %	DF %	DC/RE %	ND %
HLW	Disposal	Y	N	0.200	0.200	0.00	0.00	0.00	0.00	0.00	0.00	100.00

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Processing - Treatment method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Compaction	N	N	Same	N

Processing - Conditioning method(s)

Method	Status			
	Planned	R&D program	Current practice method use over the last 5 years	Past Practice
Bituminization	N	N		Y
Cementation	N	N	Increase	N
Encapsulation	N	N	Same	N

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Spent Sources <=30 years in Disposition

Nuclide	Number of Sources/Total Activity of Sources (GBq)			c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 4GBq	Group II more than 4GBq but less than or equal 4E+4GBq	Group III more than 4E+4GBq					
	num/activity	num/activity	num/activity					
Co-60	6261	1113		Y	Y	N	5.322E+005	
	1.950E+002	5.320E+005						
Cs-137	1859	790		N	Y	N	8.569E+003	
	4.090E+002	8.160E+003						
H-3	418	1969		Y	Y	N	1.530E+005	
	3.110E+001	1.530E+005						
Ir-192	4540	90		Y	N	N	3.240E+003	
	1.010E+001	3.230E+003						
Kr-85	6970			Y	Y	N	1.750E+002	
	1.750E+002							
Pm-147	709			N	Y	N	2.590E+001	
	2.590E+001							
Po-210	531			N	Y	N	4.570E+001	
	4.570E+001							
Sr-90	1002	311		Y	N	N	2.980E+004	
	9.530E+001	2.970E+004						
Tm-170	117			N	Y	N	7.900E-011	
	7.900E-011							

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Spent Sources > 30 years in Disposition

Nuclide	Number of Sources/Total Activity of Sources (GBq)		c o n d	u n c o n d	c a t	Total Activity for all Groups (GBq)	Decay Date
	Group I less than or equal 2 GBq	Group II more than 2GBq					
	num/activity	num/activity					
Am-241	6476	119	N	Y	N	1.870E+003	
	1.100E+002	1.760E+003					
Am-241	50	56	N	Y	N	8.163E+003	
	9.310E+001	8.070E+003					
C-14		152	Y	N	N	5.350E+003	
		5.350E+003					
Pu-238	72		N	Y	N	1.160E+002	
	1.160E+002						
Pu-238	3	3	N	Y	N	8.370E+002	
	1.040E+000	8.360E+002					
Pu-239	482	1	N	Y	N	1.229E+001	
	2.610E+000	9.680E+000					
Ra-226	1643		Y	N	N	2.250E+002	
	2.250E+002						
Ra-226	21	20	N	Y	N	1.528E+002	
	1.880E+001	1.340E+002					
Tc-99	3091		N	Y	N	1.910E+002	
	1.910E+002						